Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road, Waterford, CT 06385

Dominion®

DEC 1 0 2008

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No.	08-0726
MPS Lic/GJC	R0
Docket No.	50-423
License No.	NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 3
LICENSEE EVENT REPORT 2008-003-00
AUTOMATIC REACTOR TRIP DURING
SHUTDOWN FOR REFUELING OUTAGE 3R12

This letter forwards Licensee Event Report (LER) 2008-003-00 documenting a condition discovered at Millstone Power Station Unit 3 on October 11, 2008. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in manual or automatic actuation of systems listed in 10 CFR 50.73(a)(2)(iv)(B).

If you have any questions or require additional information, please contact Mr. William D. Bartron at (860) 444-4301.

Sincerely,

12/10/2008

Linwood L. Morris
Plant Manager – Nuclear

Attachments: 1

Commitments made in this letter: None

IE22
NRR

cc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Ms. C. J. Sanders
Project Manager
U.S. Nuclear Regulatory Commission Mail Stop 08B3
One White Flint North
11555 Rockville Pike
Rockville, MD 20852-2738

NRC Senior Resident Inspector
Millstone Power Station

ATTACHMENT

LICENSEE EVENT REPORT 2008-003-00
AUTOMATIC REACTOR TRIP DURING SHUTDOWN FOR REFUELING OUTAGE 3R12

MILLSTONE POWER STATION UNIT 3
DOMINION NUCLEAR CONNECTICUT, INC.

NRC FORM 366 (9-2007) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 EXPIRES 08/31/2010

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Millstone Power Station - Unit 3	05000423	1 OF 2

4. TITLE

Automatic Reactor Trip During Shutdown for Refueling Outage 3R12

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
10	11	2008	2008	–003–	–00	12	10	2008	FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE	1
10. POWER LEVEL	30

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: *(Check all that apply)*

20.2201(b)	20.2203(a)(3)(ii)	50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)
20.2201(d)	20.2203(a)(4)	50.73(a)(2)(iii)	50.73(a)(2)(x)
20.2203(a)(1)	50.36(c)(1)(i)(A)	X 50.73(a)(2)(iv)(A)	73.71(a)(4)
20.2203(a)(2)(i)	50.36(c)(1)(ii)(A)	50.73(a)(2)(v)(A)	73.71(a)(5)
20.2203(a)(2)(ii)	50.36(c)(2)	50.73(a)(2)(v)(B)	OTHER
20.2203(a)(2)(iii)	50.46(a)(3)(ii)	50.73(a)(2)(v)(C)	Specify in Abstract below or in NRC Form 366A
20.2203(a)(2)(iv)	50.73(a)(2)(i)(A)	50.73(a)(2)(v)(D)	
20.2203(a)(2)(v)	50.73(a)(2)(i)(B)	50.73(a)(2)(vii)	
20.2203(a)(2)(vi)	50.73(a)(2)(i)(C)	50.73(a)(2)(viii)(A)	
20.2203(a)(3)(i)	50.73(a)(2)(ii)(A)	50.73(a)(2)(viii)(B)	

12. LICENSEE CONTACT FOR THIS LER

NAME	TELEPHONE NUMBER (Include Area Code)
William D. Bartron, Supervisor Nuclear Station Licensing	860-447-1791 x4301

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED

☐ YES (If yes, complete EXPECTED SUBMISSION DATE). ☒ NO

15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR

16. ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

On October 11, 2008, while in MODE 1 at approximately 30% power an automatic reactor trip occurred, as Millstone Power Station Unit 3 was reducing power in preparation for the 12th refueling outage. Prior to the reactor trip the plant was experiencing Steam Generator (S/G) level oscillations as the result of removing feedwater system components from service. The operators took manual control of the feedwater system, however, the oscillations increased and the 'A' S/G reached its 'high-high' set point resulting in an automatic turbine trip and feedwater isolation. A reactor trip on 'low-low' S/G level followed. All safety systems operated normally including the automatic start of the auxiliary feedwater pump, and the reactor shut down safely. All secondary systems operated normally.

The cause was determined to be the operating crew on shift failed to effectively use the tools necessary to enable an event free shutdown of the plant. The organization with respect to monitoring and measuring crew performance was not effective in implementing programs designed to manage the challenges to the operators.

Prior to start up from the refueling outage, training on low power feed station operations for on shift and active licensed operators was conducted. Additionally proper operation of the S/G water level control system and the feedwater regulating valves was verified. Additional corrective actions will be entered into the station's corrective action program.

This event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in manual or automatic actuation of systems listed in 10 50.73(a)(2)(iv)(B).

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Millstone Power Station - Unit 3	05000423	2008	-- 003 --	00	2 OF 2

NARRATIVE *(If more space is required, use additional copies of NRC Form 366A)* **(17)**

1. EVENT DESCRIPTION:

On October 11, 2008, while in MODE 1 at approximately 30% power an automatic reactor [AB] [RCT] trip occurred, as Millstone Power Station Unit 3 (MPS3) was reducing power in preparation for the 12th refueling outage. Prior to the reactor trip the plant was experiencing Steam Generator (S/G) [SB] [SG] level oscillations as the result of removing feedwater system [SJ] components from service. In an effort to control the S/G level oscillations, the operators took manual control of the feedwater system; however, the oscillations increased to the point that the 'A' S/G reached its 'high-high' set point resulting in an automatic turbine [TRB] trip and feedwater isolation. Once the turbine tripped, the 'C' S/G level quickly shrank to its 'low-low' S/G level set point resulting in a reactor trip. All safety systems operated normally including the automatic start of the auxiliary feedwater (AFW) pump [BA] [P], and the reactor shut down safely. All secondary systems operated normally.

2. CAUSE:

The cause was determined to be the operating crew on shift failed to effectively use the tools necessary to enable an event free shutdown of the plant. The organization with respect to monitoring and measuring crew performance was not effective in implementing programs designed to manage the challenges to the operators.

3. ASSESSMENT OF SAFETY CONSEQUENCES:

With MPS3 at approximately 30% power during the down-power evolution, the S/G levels began to oscillate over a wider and wider range. The S/G oscillations increased to the point that the 'A' S/G reached its 'high-high' set point resulting in an automatic turbine trip and feed water isolation. Once the turbine tripped, the 'C' S/G quickly shrank to its 'low-low' S/G level set point resulting in a reactor trip. All safety systems operated normally and the reactor shut down safely.

As the result of the reactor trip, the AFW pump automatically started in accordance with plant design and the system operated as designed. No other mitigating systems were challenged.

The integrity of the fuel, reactor coolant system, or containment was not challenged or degraded as the result of this event.

There were no challenges to public or occupational radiation safety, or personnel safety as the result of this event.

4. CORRECTIVE ACTION:

Prior to start up from the refueling outage, training on low power feed station operations for on shift and active licensed operators was conducted. Additionally proper operation of the S/G water level control system and the feedwater regulating valves [FCV] was verified. Additional corrective actions will be entered into the station's corrective action program.

5. PREVIOUS OCCURRENCES:

LER 2005-005-00 documents an automatic reactor trip of Millstone Unit 3 due to 'low-low' steam generator level.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].